



INITIAL INDIVIDUAL DOSE ASSESSMENT FOLLOWING A NUCLEAR ACCIDENT

C. CHALLETON-DE VATHAIRE, E. QUENTRIC, D. DIDIER, E. BLANCHARDON, E. DAVESNE, A. RANNOU, M. AGARANDE, V. RENAUD-SALIS, D. FRANCK

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[CONTEXT

Accident on a nuclear power plant (NPP)



large amounts of radioiodine can be released

It concentrates in the thyroid and can increase the rate of thyroid cancer, as after the Chernobyl accident In the early phase of a nuclear reactor accident, *in-vivo* monitoring of impacted population would be highly useful to detect potential contamination during the passage of the cloud



Incorporated ¹³¹I will be easily detectable at the thyroid level if the measurement is performed during the few weeks following the intake



[CONTEXT

- ¹³¹I measured in the thyroid will allow calculation of the thyroid equivalent dose and of the effective dose due to the inhalation of ¹³¹I
- However, short-lived radionuclides other than ¹³¹I (¹³²Te/¹³²I, ¹³³I and ¹³⁵I), present in the release after a nuclear reactor accident, may be responsible for a significant part of the thyroid dose and/or of the effective dose received during the dispersion of the plume
- Moreover, exposition to ionizing radiation will also be due to :
- Inhalation of RN others than iodine and tellurium
- external irradiation from radionuclides present in the air (immersion)
- external irradiation from radionuclides deposited on the soil (ground shine)



[OBJECTIVE

To calculate coefficients which could be used to rapidly convert *in-vivo* ¹³¹I measurement results directly into doses, not only from inhalation of ¹³¹I but also from inhalation of all radionuclides present in the releases and from all sources of exposure

Doses

- Equivalent dose to the thyroid
- Effective dose

Pre-assessed scenarios, established by IRSN, for which **projected doses**, including those due to the inhalation of ¹³¹I, have been calculated via radioecological and dosimetric impact modelings



[METHOD



[TWO SCENARIOS OF ACCIDENT AFFECTING A NUCLEAR POWER PLANT

a loss-of-coolant accident (LOCA) leading to core meltdown

lodine isotopes are supposed to be released as aerosol or as methyl iodide

Family of radionuclides	Activity released over 24 hours (Bq)
Noble gases	$2.2 \ge 10^{16}$
Iodine	5.6 x 10 ¹³
Cesium	$1.4 \ge 10^{13}$
Tellurium	2.4 x 10 ¹³

Dispersion conditions

Parameter	Value
Releases height	at ground level
Diffusion class	weak
Wind speed at 10 m	3 m/s
Precipitation	None
Wind meandering factor	3



[TWO SCENARIOS OF ACCIDENT AFFECTING A NUCLEAR POWER PLANT

steam generator tube rupture (SGTR) accident

Family of radionuclides	Activity released over 1 hour (Bq)
Noble gases	2.0 x 10 ¹³
Iodine	2.1 x 10 ¹³
Cesium	4.0 x 10 ¹²

Iodine isotopes are considered to be released entirely in aerosol form

Dispersion conditions

Parameter	Value
Releases height	Ground level
Diffusion class	weak
Wind speed at 10 m	5 m/s
Precipitation	Rain 2 mm/h
Wind meandering factor	none



[MODEL-BASED PROJECTED DOSES THE TWO SCENARIO

- six fictive persons of the public (6 ICRP age class)
- with a breathing rate depending on its age category

stationed at a fixed location at a distance of 1 km downwind

exposed for the first 24 hours after the beginning of the radioactive releases

receiving no protection from personal equipment or shielding effect.

Doses provided :

- thyroid equivalent dose due to inhalation of all radionuclides
- effective dose due to inhalation of all radionuclides
- effective doses due to immersion in plume
- effective dose due to external irradiation from the ground
- effective dose due to all exposure pathways

[¹³¹I INTAKE FOR 1 Bq MEASURED AT THE THYROID

intakes of ¹³¹I which result in the measurement of 1 Bq of ¹³¹I at the thyroid level, I_{1131,1Bqmes} are calculated by dividing 1 Bq by the fraction of ¹³¹I, m(t), retained in the thyroid at time t as predicted by the biokinetic model

$$I_{I131,1Bqme} = \frac{1}{m(t)}$$

time t is the delay between the contamination, by inhalation, and the *in-vivo* measurement

calculation are performed

- for the six age ranges
- for 18 times t : days 1 to 10, 12, 14, 16, 18, 20, 23, 26 and 30



[RESULTS

Table 11b – Thyroid equivalent dose due to the inhalation of all radionuclides for 1 Bg of 131 I measured *in-vivo*, from the first (D1) to the thirtieth day (D30) after the accident, in the LOCA scenario (Sy).

age category	D1	D2	D3	D4	D5	D6	D7	D8	D9
3 months	2.4 x 10 ⁻⁵	2.6 x 10 ⁻⁵	2.9 x 10 ⁻⁵	3.3 x 10 ⁻⁵	3.8 x 10 ⁻⁵	4.4 x 10 ⁻⁵	5.0 x 10 ⁻⁵	5.7 x 10 ⁻⁵	6.5 x 10 ⁻⁵
1 year	2.2 x 10 ⁻⁵	2.3 x 10 ⁻⁵	2.6 x 10 ⁻⁵	3.0 x 10 ⁻⁵	3.4 x 10 ⁻⁵	3.8 x 10 ⁻⁵	4.3 x 10 ⁻⁵	4.9 x 10 ⁻⁵	5.5 x 10 ⁻⁵
5 years	1.2 x 10 ⁻⁵	1.3 x 10 ⁻⁵	1.4 x 10 ⁻⁵	1.6 x 10 ⁻⁵	1.8 x 10 ⁻⁵	2.0 x 10 ⁻⁵	2.3 x 10 ⁻⁵	2.5 x 10 ⁻⁵	2.8 x 10 ⁻⁵
10 years	5.9 x 10 ⁻⁶	6.1 x 10 ⁻⁶	6.7 x 10 ⁻⁶	7.4 x 10 ⁻⁶	8.2 x 10 ⁻⁶	9.0 x 10 ⁻⁶	9.9 x 10 ⁻⁶	1.1 x 10 ⁻⁵	1.2 x 10 ⁻⁵
15 years	3.8 x 10 ⁻⁶	4.0 x 10-6	4.3 x 10 ⁻⁶	4.8 x 10 ⁻⁶	5.2 x 10-6	5.8 x 10-6	6.4 x 10 ⁻⁶	7.0 x 10-6	7.7 x 10 ⁻⁶
adult	2.5 x 10 ⁻⁶	2.5 x 10-6	2.8 x 10 ⁻⁶	3.1 x 10 ⁻⁶	3.4 x 10 ⁻⁶	3.7 x 10-6	4.1 x 10 ⁻⁶	4.5 x 10 ⁻⁶	4.9 x 10 ⁻⁶

10 tables

age category	D10	D12	D14	D16	D18	D20	D23	D26	D30
3 months	7.4 x 10 ⁻⁵	9.6 x 10 ⁻⁵	1.2 x 10 ⁻⁴	1.6 x 10 ⁻⁴	2.1 x 10 ⁻⁴	2.7 x 10 ⁻⁴	4.1 x 10 ⁻⁴	6.0 x 10 ⁻⁴	1.0 x 10 ⁻³
1 year	6.2 x 10 ⁻⁵	7.9 x 10 ⁻⁵	1.0 x 10 ⁻⁴	1.3 x 10 ⁻⁴	1.6 x 10 ⁻⁴	2.1 x 10 ⁻⁴	3.0 x 10 ⁻⁴	4.2 x 10 ⁻⁴	6.8 x 10 ⁻⁴
5 years	3.1 x 10 ⁻⁵	3.9 x 10 ⁻⁵	4.8 x 10 ⁻⁵	6.0 x 10 ⁻⁵	7.4 x 10 ⁻⁵	9.3 x 10 ⁻⁵	1.3 x 10 ⁻⁴	1.8 x 10 ⁻⁴	2.7 x 10 ⁻⁴
10 years	1.3 x 10 ⁻⁵	1.6 x 10 ⁻⁵	1.9 x 10 ⁻⁵	2.4 x 10 ⁻⁵	2.9 x 10 ⁻⁵	3.5 x 10 ⁻⁵	4.6 x 10 ⁻⁵	6.1 x 10 ⁻⁵	8.9 x 10 ⁻⁵
15 years	8.4 x 10 ⁻⁶	1.0 x 10 ⁻⁵	1.2 x 10 ⁻⁵	1.5 x 10 ⁻⁵	1.8 x 10 ⁻⁵	2.2 x 10 ⁻⁵	2.9 x 10 ⁻⁵	3.8 x 10 ⁻⁵	5.6 x 10 ⁻⁵
adult	5.4 x 10 ⁻⁶	6.5 x 10 ⁻⁶	7.8 x 10 ⁻⁶	9.4 x 10 ⁻⁶	1.1 x 10 ⁻⁵	1.4 x 10 ⁻⁵	1.8 x 10 ⁻⁵	2.4 x 10 ⁻⁵	3.5 x 10 ⁻⁵

Challeton-de Vathaire C, Quentric E, Didier D, Blanchardon E, Davesne E, Rannou A, Agarande M, Renaud-Salis V, Franck D. INITIAL EVALUATION OF INDIVIDUAL DOSES IN THE EARLY PHASE OF A NUCLEAR REACTOR ACCIDENT BASED ON IN-VIVO MONITORING DATA AND SIMULATED RADIOLOGICAL CONSEQUENCES. Radiat Prot Dosimetry. 2019 Nov 30;185(1):96-108.



[RESULTS

SGTR accident

Contributions of the different exposure component to the dose



Effective dose due to all exposure pathways :

- 72% due to inhalation of all RN (47% due to inhalation of ¹³¹I)
- 19% due to irradiation from the ground
- 7% due to immersion in the plume



[RESULTS

Contributions of the different exposure component to the dose



LOCA accident

Effective dose due to all exposure pathways :

- 26% due to inhalation of all RN (14% due to inhalation of ¹³¹I)
- 10% due to irradiation from the ground
- 64% due to immersion in the plume



[CONCLUSION (1)

- Early interpretation of ¹³¹I *in-vivo* measurement in the thyroid, as presented in this study, will give crucial indication on the real level of exposure of the population around the nuclear site during the emergency phase
- Data provided for two scenarios of accidents affecting a nuclear power plant are directly applicable to perform a first dosimetric assessment of the exposure for a person in whose thyroid ¹³¹I has been detected and quantified in the aftermath of an accident related to one of the two scenarios considered (i.e. with similar activity releases and dispersion conditions)
- The study can be extended to other accident scenarios in the emergency preparedness phase
- During the emergency response phase of a real accident, the method can be used to create and adapt interpretation tables following the iterative assessments performed by the responders (based on modeling and available environmental measurements)

[CONCLUSION (2)

In case of a real accident a more sophisticated dose assessment will have to be performed in a second time in order to evaluate individual health risk for long term medical survey and to provide a quantification of the exposure for epidemiological studies

This assessment will take into account

- the local isotopic composition of the contaminated cloud,
- the time-pattern of the exposure,
- exposure after the first 24 hours (from the radionuclides deposited on the ground),
- subsequent individual measurements.



